

SECTION VI

NMSS HEADQUARTERS TRANSPORTATION PACKAGING AND DRY STORAGE SYSTEM SAFETY INSPECTOR NRC INSPECTOR QUALIFICATION JOURNAL

Applicability

This NRC Inspector Qualification Journal implements NRC Manual Chapter 1246, Appendix A, Section VI, by establishing the minimum training requirements for NMSS personnel assigned to perform safety inspections of transportation packaging and dry spent fuel storage system designers, fabricators and users.

The NRC Inspector Qualification Journal serves as a guideline for the development of a Program Office Qualification Journal, and establishes the minimum training requirements consistent with NRC Manual Chapter 1246. The Program Office Qualification Journal must provide traceable documentation to show that minimum requirements are met for each inspector.

The NRC Inspector Qualification Journal consists of a series of qualification guides and signature cards. Each signature card is used to document task completion, as indicated by the appropriate signature blocks. The corresponding qualification guide establishes the minimum knowledge levels or areas of study that must be completed for each signature card.

Most of the qualification guides are divided into sections. The review sections of the qualification guides identify references with general application to the inspector's qualification. The inspector is expected to have a general familiarity with these references. Other sections of the qualification guides identify specific references that have direct application to an inspection discipline. The inspector is expected to demonstrate detailed knowledge of the inspection discipline specific references.

In order to support the review of upper tier documents, programs, and policies, the inspector's First Line Supervisor will assign one or more specific reactor facilities, fuel facilities, non-power reactor facilities and/or material licensees as reference facilities. The selection of a reference facility is intended to provide the inspector's management with the ability to tailor the qualification process to the experience and training level of the inspector, and to meet the inspection needs of the NRC. The use of specific real world material will reinforce the qualification process.

Qualification Board
Requirement Met

Second Level Supervisor
or Board Chairman

Recommended as a qualified inspector

Second Level Supervisor

Certification Memo Issued

Second Level Supervisor

Qualification Card 1
NRC Orientation

A.	Site Orientation	<u>Initials</u>	<u>Date</u>
1.	New employee processing package completed	_____ Employee	_____
2.	Facility tour and introduction	_____ First Line Supervisor	_____
B.	NRC Organization		
1.	Review of NRC headquarters and NMSS organization	_____ Employee	_____
2.	Discussion of NRC organization	_____ First Line Supervisor	_____

Qualification Card 2
Code of Federal Regulations (CFR)

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	<u>Initials</u>	<u>Date</u>
A. Familiarization with selected CFR parts completed	_____ Employee	_____
B. Discussion completed on CFR parts related to transportation packaging or dry storage systems	_____ First Line Supervisor	_____

Qualification Card 3
Office Instructions

	<u>Initials</u>	<u>Date</u>
A. Familiarization with office policies and procedures	_____ Employee	_____
B. Discussion completed on office policies and procedures	_____ First Line Supervisor	_____

Qualification Card 4
Regulatory Guidance

	<u>Initials</u>	<u>Date</u>
A. Review of regulatory guidance		
1. Regulatory Guides	_____ Employee	_____
2. Information Notices/ Bulletins	_____ Employee	_____
3. NUREGs	_____ Employee	_____
4. Generic Letters	_____ Employee	_____
5. Federal Register Notices	_____ Employee	_____
6. Policy and Guidance Directives	_____ Employee	_____
B. Discussion of regulatory guidance with application to the transportation packaging and dry storage system inspection program	_____ First Line Supervisor	_____

Qualification Card 5
NRC Inspection Manual Chapters (MC)

	<u>Initials</u>	<u>Date</u>
A. Review of appropriate NRC MCs completed	_____ Employee	_____
B. Discussion of NRC MCs and their relation to the transportation packaging and dry storage system inspection program	_____ First Line Supervisor	_____

Qualification Card 6
Industry Codes and Standards

Initials

Date

A. Review of selected codes
and standards completed

Employee

B. Discussion of the application
of codes and standards to the
transportation packaging and dry
storage system inspection program

First Line Supervisor

Qualification Card 7
Inspection Accompaniments

		<u>Initials</u>	<u>Date</u>
A.	Inspections completed		
1.	_____ Facility	_____ Employee	_____
2.	_____ Facility	_____ Employee	_____
3.	_____ Facility	_____ Employee	_____
4.	_____ Facility	_____ Employee	_____
B.	Discussion of inspection and employee's role		
1.	_____ Facility	_____ First Line Supervisor	_____
2.	_____ Facility	_____ First Line Supervisor	_____
3.	_____ Facility	_____ First Line Supervisor	_____
4.	_____ Facility	_____ First Line Supervisor	_____

Qualification Card 8
NRC Management Directives

Initials

Date

A. Review of selected portions of
the NRC Management Directives
completed

Employee

B. Discussion of the application
of the NRC Management Directives
to the transportation
packaging and dry storage system
inspection program

First Line Supervisor

Qualification Card 9
Safety Analysis Report

	<u>Initials</u>	<u>Date</u>
A. Review of selected safety analysis reports completed	_____ Employee	_____
B. Discussions completed on selected safety analysis reports related to the inspection program	_____ Immediate Supervisor	_____

Qualification Card 10
Formal Training

A. CORE TRAINING:	<u>Initials</u>	<u>Date</u>
1. Fundamentals of Inspection Course (G-101)	_____	_____
	Training Coordinator	
2. Inspecting for Performance Course (G-303 or G-304)	_____	_____
	Training Coordinator	
3. Effective Communications for NRC Inspectors	_____	_____
	Training Coordinator	
4. OSHA Indoctrination Course (G-111)	_____	_____
	Training Coordinator	
5. NMSS Radiation Worker Training (H-102)]	_____	_____
	Training Coordinator	
6. Transportation of Radioactive Materials Course (H-308)	_____	_____
	Training Coordinator	

B. SPECIALIZED TRAINING

Other specialized training courses required for inspectors performing inspections in specific areas:

<u>Course Title</u>	<u>Course #</u>	<u>Initials</u>	<u>Initials</u>	<u>Date</u>
_____	_____	Supervisor	Training Coordinator	_____
_____	_____	Supervisor	Training Coordinator	_____
_____	_____	Supervisor	Training Coordinator	_____
_____	_____	Supervisor	Training Coordinator	_____

Qualification Guide 1
NRC Orientation

A. Site Orientation

1. The qualifying individual should read and complete, as appropriate, the following forms for processing into the NRC:
 - a. Personnel information
 - b. Health insurance elections
 - c. Retirement plan elections
 - d. Savings elections (e.g. U.S. Savings Bonds, TSP, etc.)
 - e. Fitness for Duty requirements and physical examination
 - f. Any other forms which may be required by NRC Office of Human Resources
 - g. Forms for issuance of tagged, controlled NRC equipment
 - h. Payroll forms and time cards
 - i. Regulatory Information Tracking System (RITS)
2. The First Line Supervisor should orient the qualifying individual to the facility as follows:
 - a. Tour the facility and introduce the qualifying individual to the staff
 - b. Indicate to the qualifying individual the location of controlled documents, reference material, supplies, office equipment, etc.

B. NRC Organization

1. The qualifying individual should review and become familiar with:
 - a. Organizational charts of region, NMSS, and headquarters and overall NRC organization (NUREG 0325)
 - b. Role of Headquarters in policy and interpretation of regulations
 - c. Role of NRC General Counsel
 - d. Role of NRC Inspector General
 - e. Role of NRC Public Affairs
 - f. Role of NRC Office of Investigations
 - g. Role of NRC Office of Enforcement

- h. Physical location of NRC offices and regions |
- i. Role of NRC as a regulatory agency |
 - (1) 10 CFR Part 1 (Organization)
 - (2) Atomic Energy Act of 1954, as amended
 - (3) Energy Reorganization Act of 1974, as amended
 - (4) NRC Enforcement Policy (NUREG 1600) |
 - (5) Incident Response Plan (NUREGs 0728 and 0845)
 - (6) Energy Policy Act of 1992
- 2. The First Line Supervisor should discuss NRC organization and role with the qualifying individual to ensure the qualifying individual has a full understanding of NRC's organization and mission and the role of the inspector in that mission.

Qualification Guide 2
Code of Federal Regulations (CFR)

A. A selection of currently applicable CFR Parts should be made by the First Line Supervisor. The selection should include the references listed below and be documented. The qualifying individual should be expected to have a general knowledge of the topics addressed in the references. This review may be accomplished by self-study, study-quizzes, briefings, or discussions.

1. 10 CFR Part 1 Statement of organization and general information
2. 10 CFR Part 2 Rules of practice for domestic licensing proceedings and issuance of orders
3. 10 CFR Part 9 Public Records
4. 10 CFR Part 19 Notices, instructions and reports to workers; inspections
5. 10 CFR Part 20 Standards for protection against radiation (includes selected Questions and Answers, Q & As)
6. 10 CFR Part 21 Reporting of defects and noncompliance
7. 10 CFR Part 25 Access authorization for licensee personnel
8. 10 CFR Part 26 Fitness for duty programs
9. 10 CFR Part 71 Packaging and transportation of radioactive material
10. 10 CFR Part 72 Licensing requirements for the independent storage of spent nuclear fuel and high-level radioactive waste
11. 10 CFR Part 150 Exemptions and continued regulatory authority in agreement states and in offshore waters under section 274
12. 10 CFR Part 170 Fees for facilities, materials, import and export licenses and other regulatory services under the Atomic Energy Act of 1954, as amended
13. 10 CFR Part 171 Annual fees for reactor operating licenses, and fuel cycle licenses and materials licenses, including holders of certificates of compliance, registrations, and quality assurance program approvals and government agencies licensed by NRC
14. 49 CFR Parts 171 through 180 Transportation

B. Following completion of the qualifying individual's self study of the listed CFR Parts, a discussion will be held with the qualifying inspector by the First Line Supervisor to test the qualifying inspector's knowledge of these Parts. To the extent possible, recent application of various sections, new regulatory initiatives, and current industry issues should be emphasized.

Qualification Guide 3
Office Instructions

A. NMSS Office Policies and Procedures

1. Read the NMSS Policy and Procedures Manual
2. The qualifying individual should review the NMSS policies and practices on:
 - a. Travel, including Management Directive 14.1 Official Temporary Duty Travel
 - b. Telephone use
 - c. Policies on use of annual leave and sick leave and excused leave, including Bulletin 4135, Leave Administration
 - d. Work schedule, including NRC Appendix 4136, Hours of Work and Premium Pay
 - e. Use of government equipment, including computers (NUDOCS and ADAMS) and Management Directive 13.1, Property Management
 - f. Union activities, including Management Directive 10.102, Labor-Management Relations Program for Federal Employees
 - g. Communications outside NRC
 - h. Policies on outside employment and acceptance of gifts
 - i. Participation in political activities
 - j. Routing of mail and procedures for sending mail and materials (via U.S. Mail, Federal Express, etc.), including Management Directive 3.23, Mail Management
 - k. Ordering of documents (e.g NUREGs)
 - l. NMSS emergency and evacuation procedures
 - m. Employee appraisal system and Individual Development Plan (IDP)
 - (1) Employee trial period (Management Directive 10.14 Employment and Staffing)
 - (2) Employee appraisals (Management Directive 10.67, Non-SES Performance Appraisal System)
 - n. Differing Professional Views or Opinions (Management Directive 10.159, General Personnel Management Provisions)

- B. The First Line Supervisor should discuss these policies and practices with the qualifying individual to ensure that the qualifying individual has a full and complete understanding.

Qualification Guide 4
Regulatory Guidance

A. A selection of currently applicable regulatory guidance should be identified by the First Line Supervisor. These references should include those listed below and should be documented. The qualifying individual should be expected to have a general knowledge of the topics addressed in the references. The review may be accomplished by self-study, study-quizzes, briefings, or discussions. Note that many Regulatory Guides reference or endorse industry codes and standards listed in Qualification Guide 6. Study of corresponding and subtier codes and standards is recommended.

1. Regulatory Guides (use latest revision)

- 1.28 Quality Assurance Requirements (Design and Construction)
- 1.33 Quality Assurance Program Requirements (Operation)
- 1.38 Quality Assurance Requirements for Packaging, Shipping, Receiving, Storage, and Handling of Items for Water-Cooled Nuclear Power Plants
- 3.44 Standard Format and Content for the Safety Analysis Report for an Independent Spent Fuel Storage Installation (Water-Basin Type)
- 3.48 Standard Format and Content for the Safety Analysis Report for an Independent Spent Fuel Storage Installation or Monitored Retrievable Storage Installation (Dry Storage)
- 3.49 Design of an Independent Spent Fuel Storage Installation (Water-Basin Type)
- 3.54 Spent Fuel Heat Generation in an Independent Spent Fuel Storage Installation
- 3.60 Design of an Independent Spent Fuel Storage Installation (Dry Storage)
- 3.61 Standard Format and Content for a Topical Safety Analysis Report for a Spent Fuel Dry Storage Cask
- 3.62 Standard Format and Content for the Safety Analysis Report for Onsite Storage of Spent Fuel Storage Casks
- 5.10 Selection and Use of Pressure Sensitive Seals on Containers for Onsite Storage of Special Nuclear Material
- 7.1 Administrative Guide for Packaging and Transporting Radioactive Material
- 7.2 Packaging and Transportation of Radioactively Contaminated Biological Materials
- 7.3 Procedures for Picking Up and Receiving Packages of Radioactive Material

7.4	Leakage Tests on Packages for Shipment of Radioactive Materials
7.5	Administrative Guide for Obtaining Exemptions from Certain NRC Requirements over Radioactive Material Shipments
7.6	Design Criteria for the Structural Analysis of Shipping Cask Containment Vessels
7.7	Administrative Guide for Verifying Compliance with Packaging Requirements for Shipments of Radioactive Materials
7.8	Load Combinations for the Structural Analysis of Shipping Casks for Radioactive Material
7.9	Standard Format and Content of Part 71 Applications for Approval of Packaging of Type B, Large Quantity, and Fissile Radioactive Material
7.10	Establishing Quality Assurance Programs for Packaging Used in the Transport of Radioactive Material
8.4	Direct and Indirect Reading Pocket Dosimeters
8.7	Instructions for Recording and Reporting Occupational Radiation Exposure Data
8.8	Information Relevant to Ensuring that Occupational Radiation Exposures at Nuclear Power Stations Will Be As Low As Reasonably Achievable
8.13	Instruction Concerning Prenatal Radiation Exposure
8.29	Instruction Concerning Risks from Occupational Radiation Exposure
2.	Information Notices(IN) and Bulletins(BL)
IN 80-025	Transportation of Pyrophoric Uranium
IN 80-032	Clarification of Certain Requirements for Exclusive-Use Shipments of Radioactive Materials
IN 81-032	Transfer and/or Disposal of Spent Generators
IN 82-024	Water Leaking from Uranium Hexafluoride Overpacks
IN 82-047	Transportation of Type A Quantities of Non-Fissile Radioactive Material
IN 83-010	Clarification of Several Aspects Relating to Use of NRC-Certified Transport Packages
IN 84-014	Highlights of Recent Transport Regulatory Revisions By DOT and NRC

IN 84-050	Clarification of Scope of Quality Assurance Programs for Transport Packages Pursuant to 10 CFR 50, Appendix B
IN 84-072	Clarification of Conditions For Waste Shipments Subject To Hydrogen Gas Generation
IN 85-046	Clarification of Several Aspects of Removable Radioactive Surface Contamination Limits for Transport Packages
IN 86-067	Portable Moisture/Density Gauges: Recent Incidents and Common Violations of Requirements for Use, Transportation, & Storage
IN 86-086	Clarification of Requirements for Fabrication and Export of Certain Previously Approved Type B Packages
IN 87-026	Cracks in Stiffening Rings on 48-inch-diameter UF6 Cylinders
IN 87-031	Blocking, Bracing, and Securing of Radioactive Materials Packages in Transportation
IN 87-033	Applicability of 10 CFR Part 21 to Nonlicensees
IN 87-047	Transportation of Radiography Devices
IN 88-016	Identifying Waste Generators in Shipments of Low-Level Waste to Land Disposal Facilities
IN 88-033	Recent Problems Involving the Model Spec 2-T Radiographic Exposure Device
IN 88-062	Recent Findings Concerning Implementation of Quality Assurance Programs by Suppliers of Transport Packages
IN 88-101	Shipment of Contaminated Equipment between Nuclear Power Stations
IN 89-027	Limitations on the Use of Waste Forms and High Integrity Containers for the Disposal of Low-Level Radioactive Waste
IN 89-074	Clarification of Transportation Requirements Applicable to Return of Spent Radiopharmacy Dosages From Users to Suppliers
IN 90-027	Clarification of Regulatory Requirements for Packaging of Uranium Hexafluoride (UF6) for Transportation
IN 90-035	Transportation of Type A Quantities of Non-Fissile Radioactive Materials
IN 91-021	Inadequate Quality Assurance Program of Vendor Supplying Safety-Related Equipment
IN 91-035	Labeling Requirements for Transporting Multi-Hazard Radioactive Materials
IN 91-039	Compliance with 10 CFR Part 21, "Reporting of Defects and Noncompliance"

IN 92-062	Emergency Response Information Requirements for Radioactive Material Shipments
IN 92-072	Employee Training and Shipper Registration Requirements for Transporting Radioactive Materials
IN 93-007	Classification of Transportation Emergencies
IN 94-047	Accuracy of Information Provided to NRC During the Licensing Process
IN 95-029	Oversight of Design and Fabrication Activities for Metal Components Used in Spent Fuel Dry Storage Systems
IN 96-040	Deficiencies in Material Dedication and Procurement Practices and in Audits of Vendors
IN 96-063	Potential Safety Issue Regarding the Shipment of Fissile Material
IN 97-020	Identification of Certain Uranium Hexafluoride Cylinders that Do Not Comply With ANSI N14.1 Fabrication Standards
IN 97-024	Failure of Packing Nuts on One-inch Uranium Hexafluoride Cylinder Valves
IN 97-042	Management Weaknesses Resulting in Failure to Comply With Shipping Requirements for Special Nuclear Material
IN 97-047	Inadequate Puncture Tests For Type B Packages Under 10 CFR 71.73(c)(3)
IN 97-051	Problems Experienced Loading and Unloading Spent Nuclear Fuel Storage and Transportation Casks
IN 97-057	Leak Testing of Packaging Used in the Transport of Radioactive Material
IN 97-086	Additional Controls for Transport of the Amersham Model No. 660 Series Radiographic Exposure Devices
IN 99-029	Authorized Contents of Spent Fuel Casks
BL 79-019	Packaging of Low-Level Radioactive Waste for Transport and Burial
BL 88-006	Actions To Be Taken for the Transportation of Model No. SPEC 2-T Radiographic Exposure Device
BL 96-04	Chemical, Galvanic, or Other Reactions in Spent Fuel Storage and Transportation Casks”
BL 97-02	Puncture Testing of Shipping Packages Under 10 CFR Part 71
	Others as selected by the First Line Supervisor

3. NUREGs (latest revision, where applicable)

NUREG 0325	USNRC Organization Charts and Functional Statements
NUREG 0383	Directory of Certificates of Compliance for Radioactive Materials Packages
NUREG 1419	Directory of Certificates of Compliance for Dry Spent Fuel Storage Casks
NUREG 1536	Standard Review Plan for Dry Cask Storage Systems
NUREG 1567	Standard Review Plan for Spent Fuel Storage Facilities
NUREG 1600	General Statement of Policy and Procedures for NRC Enforcement Actions
NUREG 1609	Standard Review Plan for Transportation Packages for Radioactive Material”
NUREG 1617	Standard Review Plan for Transportation Packages for Spent Nuclear Fuel”
NUREG/CR-3019	Recommended Welding Criteria for Use in the Fabrication of Shipping Containers for Radioactive Material
NUREG/CR-1815	Recommendations for Protecting Against Failure by Brittle Fracture in Ferritic Steel Shipping Containers up to 4 Inches Thick
NUREG/CR-3826	Recommendations for Protecting Against Failure by Brittle Fracture in Ferritic Steel Shipping Containers greater than 4 Inches Thick
NUREG/CR-3854	Fabrication Criteria for Shipping Containers
NUREG/CR 5502	Engineering Drawings for 10 CFR Part 71 Package Approvals
NUREG/CR-5717	Packaging Supplier Inspection Guide
NUREG/CR-6314	Quality Assurance Inspections for Shipping and Storage Containers
NUREG/CR-6407	Classification of Transportation Packaging and Dry Spent Fuel Storage System Components According to Importance to Safety
NUREG/BR-0195	NRC Enforcement Manual

Others as selected by the First Line Supervisor

4. Generic Letters (GL)

GL 89-02	Actions to improve the Detection of Counterfeit and Fraudulently Marketed Products
GL 91-05	Licensee Commercial-Grade Procurement and Dedication Programs

GL 91-18 Information to Licensees Regarding Two NRC Inspection Manual Sections on Resolution of Degraded and Nonconforming Conditions and on Operability |

Others as selected by the First Line Supervisor

5. Federal Register Notices
6. Policy and Guidance Directives

As selected by the First Line Supervisor

Qualification Guide 5
NRC Inspection Manual Chapters (MC)

- A. A selection of currently applicable NRC MC and Inspection Procedure (IP) references with direct application to the transportation packaging and dry storage system inspection program should be identified by the First Line Supervisor. The application of the specific references to the inspection program should be studied in detail by the qualifying individual.

1. REPORTS/COMMUNICATIONS/FOLLOW-UP

MC 0230	Morning Report
MC 0610	Inspection Reports
MC 0620	Inspection Documents and Records
MC 0720	NRC Bulletins and Information Notices
MC 0801	Inspector Feedback
MC 1120	Preliminary Notifications
IP 92701	Follow-up
IP 92703	Follow-up of Confirmatory Action Letters

2. INSPECTIONS

MC 0300	Announced and Unannounced Inspections
MC 0312	Technical Assistance for Radiation Safety Inspections at Nuclear Fuel Cycle Facilities and Materials Licensees' Sites
MC 1246	Formal Qualification Programs in Nuclear Material Safety and Safeguards Program Area
MC 2600	Fuel Cycle Facility Operational Safety and Safeguards Inspection Program

3. INTERACTIONS WITH OTHER FEDERAL AGENCIES

MC 1007	Interfacing Activities between Regional Offices of NRC and OSHA
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4. INCIDENT RESPONSE

MC 1300	Incident Response Actions - Responsibility and Authority
MC 1301	Response to Radioactive Material Incidents that Do Not Require Activation of the NRC Incident Response Plan
MC 1302	Action Levels for Radiation Exposures and Contamination Associated with Materials Events Involving Members of the Public
MC 1330	Response to Transportation Accidents Involving Radioactive Materials
IP 43001	Reactive Inspection of Nuclear Vendors

5. TRANSPORTATION AND STORAGE

MC 1330	Response to Transportation Accidents Involving Radioactive Materials
MC 2681	Physical Protection and Transport of SNM and Irradiated Fuel Inspections of Fuel Facilities
MC 2690	Inspection Program For Dry Storage of Spent Reactor Fuel at Independent Spent Fuel Storage Installations
MC 2700	Vendor Inspection Program
IP 60851	Design Control of ISFSI Components"
IP 60852	ISFSI Component Fabrication By Outside Fabricators"

IP 60853	On-Site Fabrication of Components and Construction of an ISFSI”
IP 60854	Preoperational Testing of an ISFSI”
IP 60855	Operation of an ISFSI”
IP 60856	Review of 10 CFR 72.212(b) Evaluations
IP 86001	Design, Fabrication, Testing, and Maintenance of Transportation Packagings
IP 86721	Transportation (Basic)
IP 86740	Inspection of Transportation Activities
IP 86750	Solid Radioactive Waste Management and Transportation of Radioactive Materials

6. OTHER

MC 1010	Independent Assessment and Analysis
MC 1100	Notification of Significant Meetings
MC 1201	Conduct of Employees
MC 2900	Performance Appraisal Program
IP 38703	“Commercial Grade Dedication”

- B. The First Line Supervisor will hold discussions, interviews, or oral quizzes to test the qualifying individual's knowledge and understanding of the application of the selected references to the transportation packaging and dry storage suppliers inspection program inspection program.

Qualification Guide 6
Industry Codes and Standards

A. A selection of currently applicable industry codes and standards should be identified by the First Line Supervisor. These references should include those listed below and be documented. The qualifying individual should be expected to have a general knowledge of the topics addressed in the references. This review may be accomplished by self study, study quizzes, briefings, or discussions.

1. American Society of Mechanical Engineers (ASME)

ASME/NQA-1, Quality Assurance Program Requirements for Nuclear Facilities

2. International Atomic Energy Agency (IAEA)

ST-1/IAEA Safety Series 6, Regulations for the Safe Transport of Radioactive Material

Additional standards as selected and documented by the First Line Supervisor

B. The First Line Supervisor should test the qualifying individual's knowledge of application of these codes and standards to the transportation packaging and dry storage system inspection program by discussions, interviews, or oral quizzes.

Qualification Guide 7
Inspection Accompaniments

- A. Each inspector should accompany certified inspectors on at least four inspections. At least two of these inspections should be performed at a facility other than the designated lead facility.
- B. The following is a guide for material that should be studied and discussed with the inspector in charge during these inspection accompaniments. The First Line Supervisor will discuss these items, as appropriate, following each inspection accompaniment.
1. The Inspection Program
 2. Scheduling and Preparation for Inspections
MC 0300 Announced and Unannounced Inspections
 3. Scope of Inspection
 4. Entrance/Exit Interviews
 5. Conduct of Inspection, Accumulation of Data
 6. Post-inspection Activities of Inspectors
MC 0610 Inspection Reports
MC 1100 Notification of Significant Meetings
 7. Morning Reports
MC 0230 Morning Report
 8. Non-routine Licensee Events
MC 1110 Potential Abnormal Occurrences
IP 90714 Non-routine Reporting Program
IP 43001 Reactive Inspection of Nuclear Vendors
 9. Preliminary Notification
MC 1120 Preliminary Notifications
 10. Bulletins/Information Notices
MC 0720 NRC Bulletins and Information Notices
 11. Use of Consultants of NRC
 12. Allegations and Investigations
Management Directive 8.8 Management of Allegations

13. Communication outside NRC

Management Directive 5.5 Public Affairs Program

Management Directive 3.6 Distribution of Unclassified NRC Staff/Contractor-Generated Reports

Qualification Guide 8
NRC Management Directives

A. A selection of currently applicable NRC Management Directive (MD) references should be identified by the First Line Supervisor. These references should include those listed below and be documented. The qualifying inspector should be expected to have a general knowledge of the topics addressed in the references. This review may be accomplished by self-study, study-quizzes, briefings, or discussions. The selection should include:

- | | | | |
|-----|---------------|--|--|
| 1. | NRC MD 9.1 | Organization Management | |
| 2. | NRC MD 9.29 | Organization and Function of Regional Offices | |
| 3. | NUREG 0325 | USNRC Functional Organization Chart | |
| 4. | NRC MD 3.2 | Privacy Act | |
| 5. | NRC MD 3.1 | Freedom of Information Act | |
| 6. | NRC MD 10.130 | Safety and Health Program Under the Occupational Safety and Health Act | |
| 7. | NRC MD 10.131 | Protection of NRC Employees Against Ionizing Radiation | |
| 8. | NRC MD 14.1 | Official Temporary Duty Travel | |
| 9. | NRC MD 10.159 | Differing Professional Views or Opinions | |
| 10. | NRC MD 10.42 | Hours of Work and Premium Pay | |
| 11. | NRC MD 10.43 | Time and Attendance Reporting | |
| 12. | NRC MD 10.67 | Non-SES Performance Appraisal System | |
| 13. | NRC MD 10.101 | Employee Grievances | |
| 14. | NRC MD 8.3 | NRC Incident Investigation Procedures | |
| 15. | NRC MD 8.8 | Management of Allegations | |

B. Application of the selected NRC Management Directives to the transportation packaging and dry storage system inspection program will be discussed with the qualifying individual by the First Line Supervisor to test the qualifying individual's knowledge.

Qualification Guide 9
Safety Analysis Report

- A. The inspector should become generally familiar with the Safety Analysis Report for the packaging or storage system assigned, including the independent spent fuel storage facility (if appropriate).
- B. The appropriate sections of a facility's Technical Specifications and Updated Final Safety Analysis Report (USAR) should be reviewed with an emphasis on the application of Technical Specifications in the inspection program.
- C. After reviewing a USAR, a facility Technical Specifications, and a Safety Analysis Report, the employee will be able to specifically address the application of the references to the inspection program. The employee may demonstrate their knowledge through discussions, interviews or quizzes. These discussion activities should be conducted by senior inspectors to illustrate recent application of regulatory guidance to the inspection program. Alternatively, discussions of a similar nature can be held with the inspector's Immediate Supervisor. Completion of the discussion activities must be documented.

Qualification Guide 10
Formal Training

The standards for each Training Course are provided in the NRC Technical Training Division Course Catalog and will not be duplicated in the Qualification Guide.